

# Comparison with simulations to experimental data for photo-neutron reactions using SPring-8 Injector

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**Abstract.** Simulations of photo-nuclear reactions by using Monte Carlo codes PHITS and FLUKA have been performed to compare to the measured data at the SPring-8 injector with 250MeV and 961MeV electrons. Measurement data of Bismuth-206 productions due to photo-nuclear reactions of  $^{209}\text{Bi}(\gamma, 3n)^{206}\text{Bi}$  and high energy neutron reactions of  $^{209}\text{Bi}(n, 4n)^{206}\text{Bi}$  at the beam dumps have been compared with the simulations. Neutron leakage spectra outside the shield wall are also compared between experiments and simulations.

## 1 Introduction

Since 1997, the 8GeV class synchrotron radiation facility, SPring-8, has been operated successfully for the users. The shielding designs of the SPring-8 accelerators were performed by using Jenkins' formula [1] for the bulk shielding in the lateral direction and modified Swanson's formula [2] in the forward direction. To verify the verification of the shielding design, benchmark experiments were performed by using Helium-3 high sensitive neutron counter with polyethylene moderators, TLD dose meters and Polymeric track detectors. The dose distributions within the shield tunnel were also measured by using LiF Thermo-Luminescence detectors and Polymetric neutron track detectors ( CR-39 ) and the data were presented at ICRS9 [3]. The high energy neutron distributions around the beam dump of the accelerated electron are also measured by using bismuth detectors. Those experimental data were presented and compared with the empirical formula and EGS4 [4] with photo-neutron production cross section data [5].

In addition to FLUKA [6,7], the Monte Carlo code PHITS2.82 [8,9] have been improved recently to simulate the photo-nuclear reaction calculation. These simulation codes are much useful for radiation shielding design of electron accelerator machines as well as proton accelerators with good usability. The neutrons and gamma dose distributions around the beam dumps of the SPring-8 injector have been simulated by using both Monte Carlo codes. However experimental data, especially photonuclear reaction data are quite insufficient to compare to the simulations.

Bismuth is a useful metal to measure high energy neutrons and gamma-rays because of the neutron and gamma energy dependence of the nuclear production of the bismuth radioisotopes. At ICRS9, the data of bismuth-

206 due to 250MeV electrons were presented. In addition to the data produced by 250 MeV electrons, the experimental data produced by 961 MeV electrons are presented and both data have been compared to simulations. The neutron spectra outside the shield tunnel are compared between the PHITS simulations and experimental data.

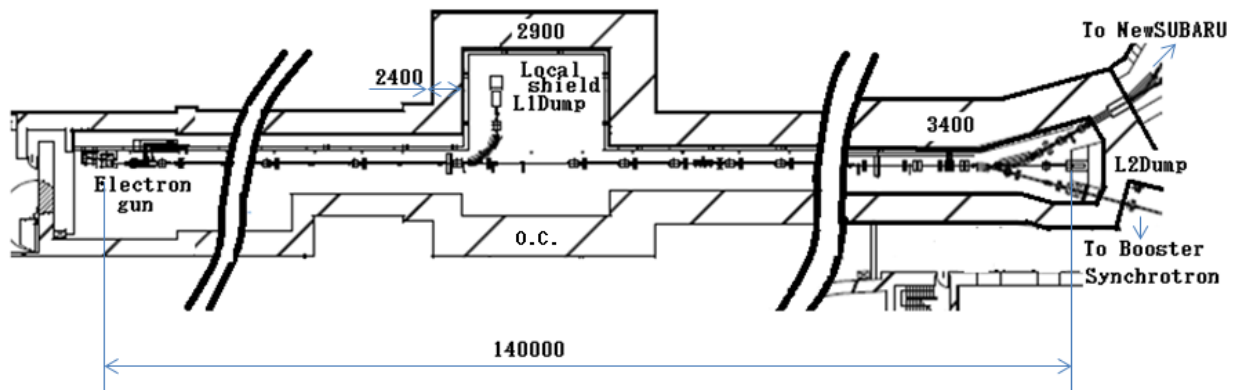
## 2 Experiments

The measurements were performed in 1998. At that time, the injector of SPring-8 had two dumps, one was 250MeV (L1) dump and the other was 1.15GeV (L2) dump. The geometrical configuration is shown in Fig.1. At the experiments, the 250MeV and 961 MeV electron beam that lead to L1 dump and L2 dump were employed and the data are indicated in Table 1, respectively. The beam dumps have double cylindrical structures, made of graphite for the inner layer and iron for the outer layer to reduce photo-neutron production and shielding.

The neutron leakage spectra outside the shield tunnel were measured by using extremely high sensitive helium-3 counter with polyethylene moderators, and the data were unfolded with SAND-II code [10]. Two types (small and large) of bismuth ingot were set around the beam dumps to measure the high energy neutrons and gamma-rays.

**Table 1.** Electron beam conditions of the linac injector during the experiments.

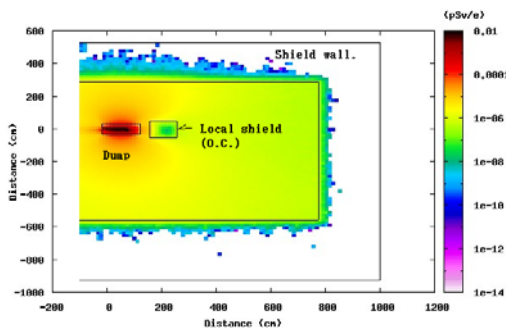
Dump	250MeV (L1dump)	1.15GeV (L2dump)
Electron energy	250MeV	961MeV
Frequency	60pps	60pps
Peak current (40ns)	143.0mA	134.3mA
Total electrons	$3.48 \times 10^6$	$7.75 \times 10^{15}$



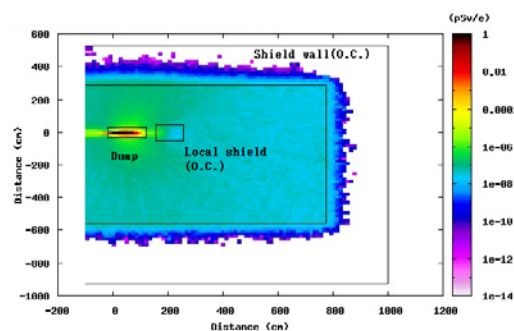
**Fig. 1.** Conceptual plane view of SPring-8 injector. (L1Dump and L2Dump are 250MeV dump and 1.15GeV dump, respectively. Unit is mm) At the end station of the linac, accelerated electrons can be extracted to 3 ways. One is to L2 dump, one is to booster synchrotron, and the other is to small SR ring, NewSUBARU.

### 3 Simulations

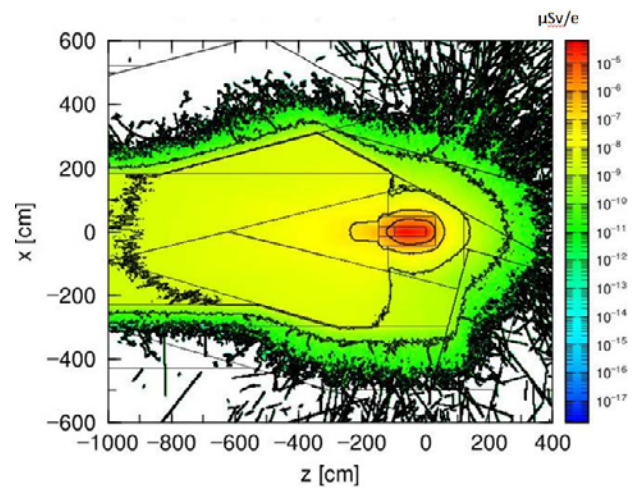
Neutron and gamma effective dose (Worst geometry) simulations around the L1 dump by using FLUKA are shown in Figs.2 and 3, respectively. PHITS simulations for effective dose distribution (AP geometry) due to 961 MeV are shown in Figs.4 and 5 for neutron and gamma doses, respectively.



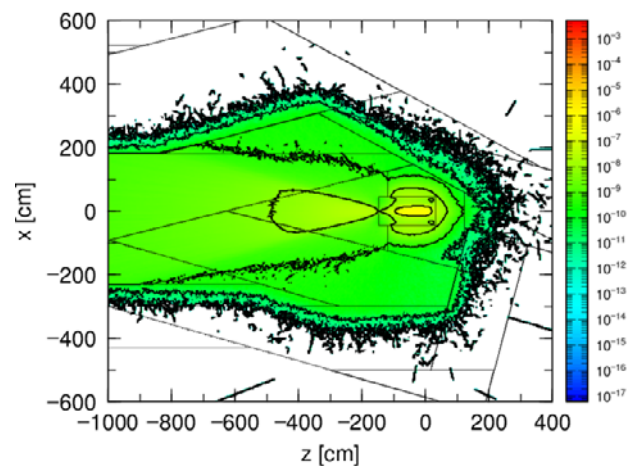
**Fig.2.** Neutron dose distribution at L1 dump (Horizontal plan at electron beam level, unit:pSv/e)



**Fig.3.** Gamma dose distribution at L1 dump. (Horizontal plan at electron beam level, unit:pSv/e)



**Fig.4.** Neutron dose distribution due to 961MeV electrons. (Horizontal plane at electron beam level, unit:μSv/e)

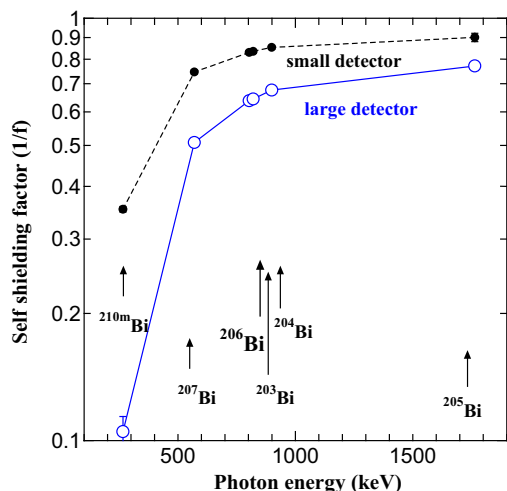


**Fig.5.** Gamma dose distribution due to 961MeV electrons. (Horizontal plane at electron beam level, unit:μSv/e)

## 4 Comparison between simulations and experiments

### 4.1 Bismuth isotopes production around the beam dumps

Small and large size bismuth ingots were employed for activation detectors and gamma-rays from bismuth detectors were measured after irradiation. The size of small detectors is 1.12 cm and 2.9 cm for the thickness and the diameter, respectively. The large detectors are 2.3 cm thick and 8 cm in diameter. In those cases, the self-shielding effect of detectors cannot be ignored to measure the gamma-rays. Figure 6 shows the self-shielding factors depending on gamma-ray energy and the size of bismuth detectors. These are PHITS simulations on the assumption of the isotropic distribution of the radioisotopes within the detectors. The assumption of the isotropic distribution is reasonable because of not so much thick detectors for high energy photons and neutrons and it's confirmed by using PHITS simulations. The previous data [3] were without



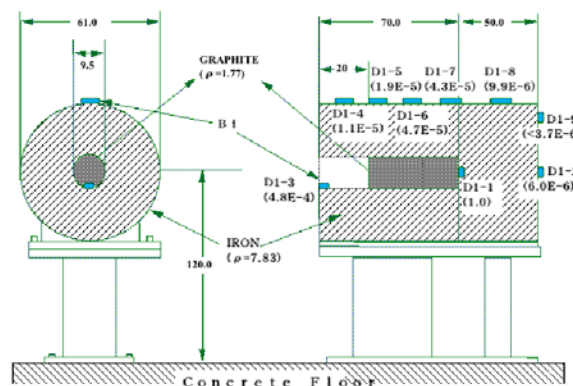
**Fig.6.** Self-shielding effect for the gamma-ray measurements of the bismuth radioisotopes depending on the photon energy and size of the detectors. The arrows of

bismuth radioisotopes indicate the photon energy of main emissions. Full and open circles are for small and large detectors, respectively.

considering these self-shielding effects so that these data must be corrected, especially for large detectors. The PHITS simulations of bismuth isotopes productions have been performed using [T-Dchain] command and DCHAIN-SP [11] with weighting factor  $p_{\text{numul}}=100$  of photo-nuclear reaction.

#### 4.1.1 250MeV electrons (L1 dump)

The bismuth-206 production distribution around the L1 dump was measured and compared to the Monte Carlo simulations. The bismuth detectors were set up as shown in Fig.7. In the experimental paper [3], the bismuth-206 production calculations were performed by using EGS4 with the library for  $^{209}\text{Bi}(\gamma, 3n)^{206}\text{Bi}$  reaction created by themselves using MCPHOTO [12] + PICA [13]. The reference paper also considered for  $^{209}\text{Bi}(n, 4n)^{206}\text{Bi}$  reaction by using ENDF-B/VI library.



**Fig.7.** Illustration of the 250 MeV electron beam dump (L1) and the location of the bismuth detectors. D1-1 to D1-9 are bismuth detectors and the figures in parenthesis under the detector number are the relative values of produced bismuth-206, which are normalized to the D1-1 detector values [3].

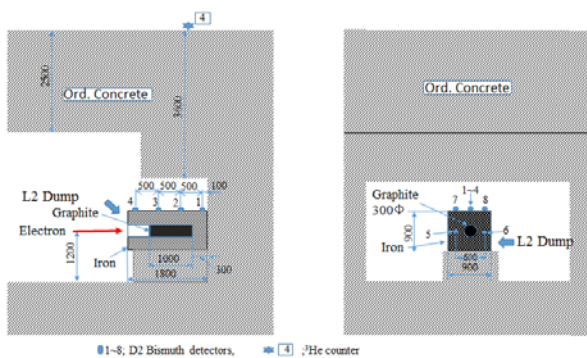
**Table 2.** Bismuth-206 production distribution due to 250MeV electron injection into dump. The errors are considered only for statistical errors. The figures in the parentheses are pro forma amounts because of large statistical errors. (\*; without considering self-shielding, \*2; with considering self-shielding, \*3; unit is converted from ( $\text{g}^{-1} \text{electron}^{-1}$ ) [3] to  $(\text{Bq/g})/(\text{e/s})$ )

Bi detector	Measured data* (Bq/g)/(e/s)	Measured data*2 (Bq/g)/(e/s)	EGS4 cal. *3 (Bq/g)/(e/s)	FLUKA cal. (Bq/g)/(e/s)	PHITS cal. (Bq/g)/(e/s)
D1-1(Small)	$1.45 \times 10^{-7} \pm 0.36\%$	$1.74 \times 10^{-7} \pm 0.36\%$	$3.7 \times 10^{-7}$	$2.27 \times 10^{-7} \pm 0.50\%$	$2.21 \times 10^{-7} \pm 1.2\%$
D1-2(Small)	$8.47 \times 10^{-13} \pm 25\%$	$1.02 \times 10^{-12} \pm 25\%$	$6.0 \times 10^{-12}$	$(6.65 \times 10^{-12} \pm 14\%)$	-
D1-3(Small)	$6.92 \times 10^{-11} \pm 2.3\%$	$8.34 \times 10^{-11} \pm 2.3\%$	$2.8 \times 10^{-10}$	$(1.40 \times 10^{-11} \pm 9.6\%)$	$(1.70 \times 10^{-10} \pm 40\%)$
D1-4(Large)	$1.61 \times 10^{-12} \pm 2.9\%$	$2.52 \times 10^{-12} \pm 2.9\%$	$1.2 \times 10^{-12}$	$(2.60 \times 10^{-12} \pm 14\%)$	-
D1-5(Large)	$2.75 \times 10^{-12} \pm 2.2\%$	$4.31 \times 10^{-12} \pm 2.2\%$	$6.2 \times 10^{-12}$	$5.03 \times 10^{-12} \pm 3.7\%$	$(2.38 \times 10^{-10} \pm 23\%)$
D1-6(Large)	$6.79 \times 10^{-12} \pm 1.4\%$	$1.06 \times 10^{-11} \pm 1.4\%$	$1.4 \times 10^{-12}$	$1.30 \times 10^{-11} \pm 2.7\%$	-
D1-7(Large)	$6.21 \times 10^{-12} \pm 1.5\%$	$9.74 \times 10^{-12} \pm 1.5\%$	$7.5 \times 10^{-12}$	$1.30 \times 10^{-11} \pm 4.5\%$	$(1.36 \times 10^{-10} \pm 41\%)$
D1-8(Large)	$1.40 \times 10^{-12} \pm 3.1\%$	$2.19 \times 10^{-12} \pm 3.1\%$	$2.9 \times 10^{-12}$	$(1.78 \times 10^{-12} \pm 14\%)$	-
D1-9(Small)	$< 5.35 \times 10^{-13}$	$< 6.45 \times 10^{-13}$	$5.2 \times 10^{-14}$	-	-

Table 2 indicates the results of the FLUKA and PHITS simulations together with the measurement data and the EGS4 calculations. In the simulations, FLUKA indicates about 1.30 times higher than that of the measurement data of D1-1 detector. The FLUKA simulation shows close agreement with the PHITS results for the bismuth-206 production within the D1-1 detector. For another detectors except D1-2 and D1-3, the FLUKA simulations show good agreement within about 30% discrepancies with the measured data. Large statistical errors are indicated for the D1-2, D1-3 detectors, and the PHITS simulations have also large statistical errors for another detectors. Experimental data of another bismuth radioisotopes with the self shielding corrections are shown in Table 3 including PHITS simulations for D1-1 detector. In Table 3, «unknown» means that a long decay time was needed to measure the gamma-rays because of very high activities. Therefore, the information of bismuth-204 and 203 was cleared.

**Table 3.** Bismuth radioisotopes production within D1-1 and D1-6 detectors. (\* ; measurement data)

Bi-det. No	Exp.(D1-1)* (Bq/g)/(e/s)	PHITS Cal (D1-1) (Bq/g)/(e/s)	EXP.(D1-6)* (Bq/g)/(e/s)
$^{207}\text{Bi}$	$2.68 \times 10^{-6}$ ±6.4%	$3.66 \times 10^{-6}$ ±0.22%	$< 2.37 \times 10^{-10}$
$^{205}\text{Bi}$	$1.01 \times 10^{-7}$ ±1.0%	$2.52 \times 10^{-8}$ ±1.4%	$9.95 \times 10^{-12}$ ± 4.8%
$^{204}\text{Bi}$	unknown	-	$8.94 \times 10^{-13}$ ± 16%
$^{203}\text{Bi}$	unknown	-	$1.24 \times 10^{-12}$ ± 31%

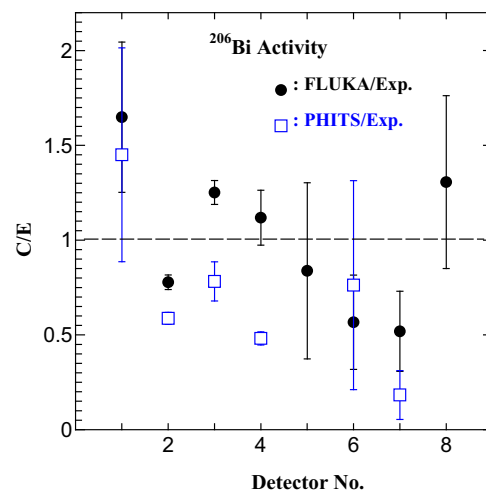


**Fig.8.** Configuration of bismuth detectors around the L2 beam dump (Vertical plane, left figure is parallel and right is perpendicular against the electron beam axis)

#### 4.1.2 961MeV electrons (L2 dump)

The bismuth-206 production distribution around the L2 dump using 961 MeV electrons was measured as well as 250MeV electrons, and the configuration of bismuth detectors are as shown in Fig.8. The measurement data and simulation results are listed in Table 4. Both simulations show agreement within about ±50 % as shown in Fig.9 even though simulations have large statistical errors. Another bismuth radioisotopes which

were produced in the detectors of D2-2, D2-3, and D2-4 are listed in Table 5 including the PHITS simulations for D2-3 detector. The measurement data of Tables 2 to 5 are useful to estimate the photon and neutron energy distributions because of the threshold energy dependence on the photon and neutron energy for each reaction such as 22.44MeV, 29.48MeV, 37.97MeV, and 45.16MeV for  $^{209}\text{Bi}(\gamma, 3n)^{206}\text{Bi}$ ,  $^{209}\text{Bi}(\gamma, 4n)^{205}\text{Bi}$ ,  $^{209}\text{Bi}(\gamma, 5n)^{204}\text{Bi}$ , and  $^{209}\text{Bi}(\gamma, 6n)^{203}\text{Bi}$ , respectively. It is difficult to estimate the amount of  $^{210}\text{Bi}$  experimentally and directly, however it is easy to compare between the simulations. In comparison with the simulations,  $^{210}\text{Bi}$  productions are listed as indicated in Table 6, and shown in good agreements.



**Fig.9.** Ratios of simulations to experimental data (Full circles and open squares are the data of FLUKA and PHITS, respectively. The errors are only considering statistical errors.)

#### 4.2 Leakage neutron spectrum outside the roof of shield tunnel

Leakage neutron spectra have been simulated by using PHITS with the [T-Point] tally of which algorithm is same as the Point Detector of MCNP5 [14] because of strong attenuation through thick shield walls. In these simulations, two points have been performed to compare the experimental data [3]. One is located directly above the L1 dump outside the roof of the tunnel with the shield thickness of 190 cm ordinary concrete, and the other is located directly above the L2 dump outside the roof of the tunnel with the shield thickness of 360 cm ordinary concrete as indicated in Figs. 1 and 8. These points are under air conditioning so that it can be avoided the change of the environmental conditions, relatively. In the simulations, the density of 2.26 g/cm<sup>3</sup> has been employed for ordinary concrete. Figs. 10 and 11 show the simulation results in comparison with the experimental data that were measured by using extremely high sensitive helium-3 counter with multipolyethylene moderators and unfolded the measurement data. These simulations show reasonably agree with the experimental data. However, the results of the simulation for the 250MeV electron case indicate slightly high in the region of high energy in comparison with the



experimental data as shown in Fig.10. On the other hand, Fig. 11 shows that the simulation results for the 961 MeV

electron case indicate the slightly lower spectrum than that of experimental data in the region of intermediate

**Table 4** Bismuth-206 production distribution due to 961MeV electron injection into L2 dump. The errors are considered only for statistical errors. The figures in the parentheses are pro forma amounts because of large statistical errors. (\*; without considering self-shielding, \*2; with considering self-shielding, \*3;unit is converted from ( $\text{g}^{-1} \text{electron}^{-1}$ ) [3] to (Bq/g)/(e/s))

Bi detector No.	Measured data* (Bq/g)/(e/s)	Measured data* <sup>2</sup> (Bq/g)/(e/s)	FLUKA cal. (Bq/g)/(e/s)	PHITS cal. (Bq/g)/(e/s)	C/E (FLUKA)	C/E (PHITS)
D2-1(Small)	$1.08 \times 10^{-11} \pm 17\%$	$1.31 \times 10^{-11} \pm 17\%$	$(2.16 \times 10^{-11} \pm 17\%)$	$(1.90 \times 10^{-11} \pm 35\%)$	$(1.7 \pm 24\%)$	$(1.5 \pm 39\%)$
D2-2(Large)	$2.01 \times 10^{-11} \pm 1.8\%$	$3.15 \times 10^{-11} \pm 1.8\%$	$2.45 \times 10^{-11} \pm 4.6\%$	$1.85 \times 10^{-11} \pm 5.0\%$	$0.78 \pm 4.9\%$	$0.59 \pm 5.3\%$
D2-3(Large)	$1.14 \times 10^{-11} \pm 2.3\%$	$1.79 \times 10^{-11} \pm 2.3\%$	$2.24 \times 10^{-11} \pm 4.5\%$	$(1.40 \times 10^{-11} \pm 13\%)$	$1.3 \pm 5.1\%$	$(0.78 \pm 13\%)$
D2-4(Large)	$1.56 \times 10^{-12} \pm 6.8\%$	$2.45 \times 10^{-12} \pm 6.8\%$	$2.74 \times 10^{-12} \pm 11\%$	$1.18 \times 10^{-12} \pm 2.2\%$	$1.1 \pm 13\%$	$0.48 \pm 7.1\%$
D2-5(Small)	$3.50 \times 10^{-12} \pm 24\%$	$4.26 \times 10^{-12} \pm 24\%$	$(3.57 \times 10^{-12} \pm 50\%)$	-	$(0.84 \pm 55\%)$	-
D2-6(Small)	$5.22 \times 10^{-12} \pm 18\%$	$6.28 \times 10^{-12} \pm 18\%$	$(3.56 \times 10^{-12} \pm 40\%)$	$(4.79 \times 10^{-12} \pm 70\%)$	$(0.57 \pm 44\%)$	$(0.76 \pm 72\%)$
D2-7(Large)	$1.09 \times 10^{-12} \pm 7.8\%$	$1.71 \times 10^{-12} \pm 7.8\%$	$(8.87 \times 10^{-13} \pm 40\%)$	$(3.13 \times 10^{-13} \pm 13\%)$	$(0.52 \pm 41\%)$	$(0.18 \pm 70\%)$
D2-8(Large)	$1.02 \times 10^{-12} \pm 8.0\%$	$1.60 \times 10^{-12} \pm 8.0\%$	$2.09 \times 10^{-12} \pm 34\%$	$(1.40 \times 10^{-11} \pm 13\%)$	$(1.31 \pm 35\%)$	-

**Table 5** Bismuth radioisotopes production within D2-2, 3, and 4 detectors. (uni ;(Bq/g)/(e/s), \* ; measurement data)

Bi	(D2-2)*	(D2-3)*	PHITS Cal. (D2-3)	(D2-4)*
<sup>205</sup> Bi	$3.22 \times 10^{-11} \pm 6.6\%$	$1.86 \times 10^{-11} \pm 8.2\%$	$1.52 \times 10^{-11} \pm 36\%$	$3.44 \times 10^{-12} \pm 18\%$
<sup>204</sup> Bi	$1.89 \times 10^{-11} \pm 3.1\%$	$9.13 \times 10^{-12} \pm 4.0\%$	$1.16 \times 10^{-11} \pm 46\%$	$1.52 \times 10^{-12} \pm 13\%$
<sup>203</sup> Bi	$1.65 \times 10^{-11} \pm 6.1\%$	$9.51 \times 10^{-12} \pm 8.4\%$	$3.48 \times 10^{-12} \pm 17\%$	$6.42 \times 10^{-13} \pm 55\%$

**Table 6** Simulation of bismuth-210 production

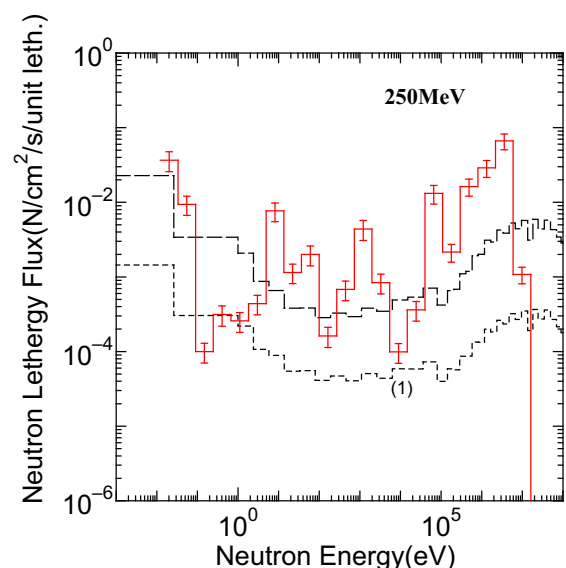
Det. No	FLUKA cal. (Bq/g)/(e/s)	PHITS cal. (Bq/g)/(e/s)	FLUKA cal PHITS cal.
No.1	$5.64 \times 10^{-11} \pm 1.7\%$	$4.22 \times 10^{-11} \pm 1.9\%$	$1.3_4 \pm 2.6\%$
No.2	$3.34 \times 10^{-11} \pm 1.6\%$	$4.34 \times 10^{-11} \pm 1.0\%$	$0.77_0 \pm 1.9\%$
No.3	$3.04 \times 10^{-11} \pm 0.88\%$	$2.97 \times 10^{-11} \pm 0.85\%$	$1.0_2 \pm 1.2\%$
No.4	$9.99 \times 10^{-12} \pm 1.9\%$	$1.05 \times 10^{-11} \pm 3.2\%$	$0.95_1 \pm 3.7\%$
No.5	$2.64 \times 10^{-12} \pm 6.4\%$	$2.67 \times 10^{-12} \pm 1.7\%$	$0.98_9 \pm 6.6\%$
No.6	$2.74 \times 10^{-12} \pm 9.6\%$	$3.24 \times 10^{-12} \pm 2.3\%$	$0.84_6 \pm 9.9\%$
No.7	$8.60 \times 10^{-12} \pm 2.5\%$	$8.11 \times 10^{-12} \pm 6.1\%$	$1.0_6 \pm 6.6\%$
No.8	$8.24 \times 10^{-12} \pm 2.0\%$	$7.70 \times 10^{-12} \pm 4.1\%$	$1.0_7 \pm 4.6\%$

energy region. These discrepancies need future investigations as well as the improvements of the statistical errors.

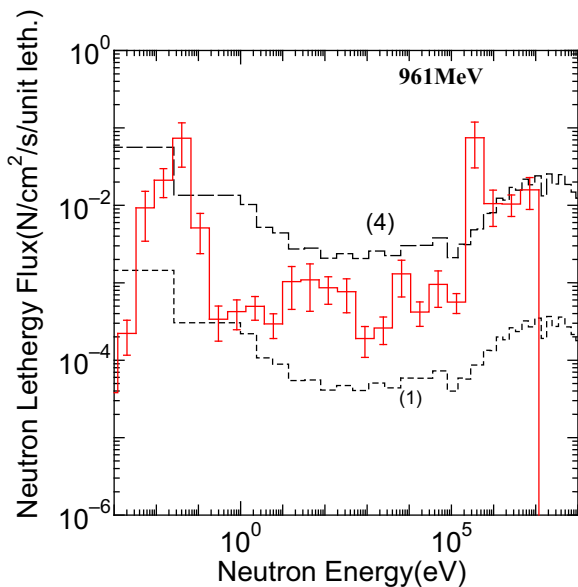
## 5 Summary

The experimental data of bismuth radioisotopes distribution at around the beam dumps produced by 961 MeV electrons are presented as well as that of due to 250

MeV electrons. These data are useful to compare to the simulations. FLUKA and PHITS Monte Carlo simulations have been performed to compare to these data and the both simulations show reasonably agreements in a range of within about  $\pm 50\%$ . The PHITS simulations are performed for the leakage neutron energy spectra to compare to the experimental data, and these show slightly different shapes. These discrepancies need future investigations.



**Fig.10** Comparison of the leakage neutron energy spectrum at the roof outside the shield tunnel and directly above of the L1 dump. (Solid red line indicates the simulation results, and dashed line is the experimental data. Dotted line indicates the neutron back ground spectrum)



**Fig.11** Comparison of the leakage neutron energy spectrum at the roof outside the shield tunnel and directly above of the L2 dump. Lines indicate the same as Fig.10.

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