

Photoneutron Reaction Data for Nuclear Physics and Astrophysics

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Abstract. We discuss the role of photoneutron reaction data in nuclear physics and astrophysics in conjunction with the Coordinated Research Project of the International Atomic Energy Agency with the code F41032 (IAEA-CRP F41032).

1 PHOENIX Collaboration for the IAEA-CRP F41032

The Coordinated Research Project of the International Atomic Energy Agency with the code F41032 (IAEA-CRP F41032) was launched in 2016 as a 5-years** project. The goal of the project is to publish two compilations of an updated photonuclear data library and a reference database of photon strength functions in 2020.

We have established an experimental collaboration called the PHOENIX (PHOTO-Excitation and Neutron emission cross [X] sections) Collaboration for the IAEA-CRP in the γ -ray beam line GACKO (Gamma Collaboration hutch of KOnan university) of the NewSUBARU synchrotron radiation facility in Japan [1]. The PHOENIX Collaboration is hosted by the Konan University; the collaboration involves the University of Oslo, ELI-NP (Extreme Light Infrastructure – Nuclear Physics), Moscow State University, Shanghai Institute of Applied Physics, Texas A&M University, Technische Universität Darmstadt, Université Libre de Bruxelles, and University of Hyogo.

In this collaboration, we are measuring (γ, xn) cross sections with $x=1-4$ for the first compilation (photonuclear data) and $(\gamma, 1n)$ cross sections near one-neutron threshold for the second compilation (photon

strength functions). A list of nuclei to be measured in the PHOENIX Collaboration is given in Table 1.

Table 1. Nuclei covered by the PHOENIX Collaboration.

	Photonuclear data	Photon strength functions
2015	²⁰⁹ Bi, ⁹ Be	²⁰⁵ Tl, ²⁰³ Tl, ⁸⁹ Y
2016	¹⁹⁷ Au, ¹⁶⁹ Tm, ⁸⁹ Y	¹⁹² Os, ¹⁸⁵ Re, ¹³⁸ Ba, ¹³⁷ Ba, ⁶⁴ Ni, ⁶¹ Ni, ⁶⁰ Ni, ⁵⁸ Ni, ¹³ C
2017	¹⁸¹ Ta, ¹⁶⁵ Ho, ⁵⁹ Co	¹⁸⁴ W, ¹⁸³ W, ¹⁸² W, ⁶⁸ Zn, ⁶⁶ Zn, ⁶⁴ Zn
2018	¹⁵⁹ Tb, ¹³⁹ La, ¹⁰³ Rh	¹⁶⁰ Gd, ¹⁵⁸ Gd, ¹⁵⁷ Gd, ¹⁵⁶ Gd

We provide (γ, xn) cross section data to resolve the long-standing discrepancy between the Livermore and Saclay data of partial photoneutron cross sections. In many cases, Saclay provided larger cross sections for $(\gamma, 1n)$ reactions than Livermore and vice versa for $(\gamma, 2n)$ reactions [2]. The ratio of the Saclay data to the Livermore data of integrated total cross sections, $R^{int} = \sigma_S^{int} / \sigma_L^{int}$, varies from 0.65 to 1.35 over 19 nuclei from ⁵¹V to ²³⁸U. As shown in Fig. 1, the ratio for

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** The data acquisition is limited to 3 years for data evaluations and model calculations.

integrated partial cross sections averaged over the 19 nuclei is very different; it is $\langle R^{\text{int}}(1n) \rangle = 1.07$ and $\langle R^{\text{int}}(2n) \rangle = 0.84$ for the $(\gamma, 1n)$ and $(\gamma, 2n)$ reactions, respectively [3].

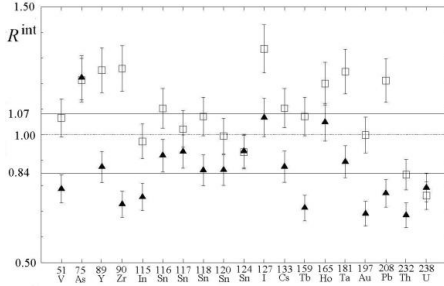


Fig. 1 The ratio of the Saclay data to the Livermore data of integrated cross sections, $R^{\text{int}} = \sigma_S^{\text{int}} / \sigma_L^{\text{int}}$, for $(\gamma, 1n)$ (open squares) and $(\gamma, 2n)$ (solid triangles) reactions.

We also provide $(\gamma, 1n)$ cross section for specific nuclei near the one-neutron threshold for an absolute normalization of the photon strength function below neutron threshold obtained with the Oslo method [4]. Figure 2 shows the resulting photon strength function for ^{117}Sn constructed experimentally, showing the presence of an extra strength over the neutron threshold at 6.94 MeV that is attributed to a pygmy E1 resonance [5].

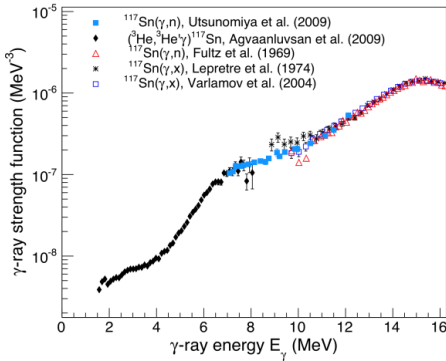


Fig. 2 (color online) Photon strength function for ^{117}Sn constructed from the photon neutron data and the data of the Oslo method.

We also provide $(\gamma, 1n)$ cross sections systematically over an isotopic chain to reach a unified understanding of (γ, n) and (n, γ) cross sections with the γ -ray strength function method [6,7]. This method effectively deduces (n, γ) cross sections for radioactive nuclei involved in the isotopic chains as a result of this unified and comprehensive approach.

The PHOENIX Collaboration will continue after the IAEA-CRP. Nuclei for which (γ, xn) cross sections will be measured after 2018 may include ^{112}Sn , ^{116}Sn , ^{120}Sn , ^{124}Sn , ^{208}Pb , ^{232}Th , and ^{238}U .

2 Direct Neutron-Multiplicity Sorting with a Flat-Response Detector

We have long used a high-efficiency neutron detector of Livermore type to measure $(\gamma, 1n)$ cross sections near the one-neutron threshold. This detector consisting of three rings of total twenty ^3He proportional counters embedded in a polyethylene moderator has a strong dependence of the detection efficiency on neutron kinetic energies. We use the so-called ring-ratio (RR) technique developed at Livermore [8] to determine the mean kinetic energies of neutrons. However, because of the RR technique, we limited ourselves to $(\gamma, 1n)$ cross section measurements below the two-neutron threshold, where the number of detected neutrons is equal to the number of $(\gamma, 1n)$ reactions. In the one-neutron channel, the ring ratio can surely determine the mean kinetic energies of neutrons produced in the $(\gamma, 1n)$ reaction. However, when we go above the two-neutron threshold, we immediately encounter an essential problem with the RR technique because there is no way to know in what reaction neutrons are produced and detected.

Let us consider measurements of (γ, xn) cross sections with neutron-multiplicity $x=1-3$ with a neutron detector of Livermore type by using a pulsed γ -ray beam. The frequency of the pulsed beam is typically 1ms because of the long neutron-moderation time of the polyethylene. During the 1ms time-interval between two successive γ -ray pulses, we detect neutrons. According to the number of detected neutrons (n), let us name neutron events as single (N_1), double (N_2), and triple (N_3) events corresponding to $n=1, 2$, and 3, respectively. The neutron events are expressed as follows.

$$\begin{aligned}
 N_1 &= N_1 \cdot \varepsilon(E_1) + \\
 N_2 &\cdot \varepsilon(E_{21}) \cdot (1 - \varepsilon(E_{22})) + \\
 N_2 &\cdot \varepsilon(E_{22}) \cdot (1 - \varepsilon(E_{21})) + \\
 N_3 &\cdot \varepsilon(E_{31}) \cdot (1 - \varepsilon(E_{32})) \cdot (1 - \varepsilon(E_{33})) + \\
 N_3 &\cdot \varepsilon(E_{32}) \cdot (1 - \varepsilon(E_{31})) \cdot (1 - \varepsilon(E_{33})) + \\
 N_3 &\cdot \varepsilon(E_{33}) \cdot (1 - \varepsilon(E_{31})) \cdot (1 - \varepsilon(E_{32}))
 \end{aligned} \tag{1}$$

$$\begin{aligned}
 N_d &= N_2 \cdot \varepsilon(E_{21})^2 + \\
 N_3 &\cdot \varepsilon(E_{31}) \cdot \varepsilon(E_{32}) \cdot (1 - \varepsilon(E_{33})) + \\
 N_3 &\cdot \varepsilon(E_{31}) \cdot \varepsilon(E_{33}) \cdot (1 - \varepsilon(E_{32})) + \\
 N_3 &\cdot \varepsilon(E_{32}) \cdot \varepsilon(E_{33}) \cdot (1 - \varepsilon(E_{31}))
 \end{aligned} \tag{2}$$

$$N_t = N_3 \cdot \varepsilon(E_{31}) \cdot \varepsilon(E_{32}) \cdot \varepsilon(E_{33}) \tag{3}$$

Here N_x ($x=1,2,3$) is the number of (γ, xn) reactions induced by γ -ray pulses and E_{ij} is the kinetic energy of the j -th neutron emitted in the (γ, in) reaction. Note that the neutron detection efficiency (ε) depends on the neutron kinetic energy.

In the triple neutron event, obviously all of the first-, second-, and third-emitting neutrons are detected when the $(\gamma, 3n)$ reaction takes place. The second term of the

double neutron event means that the first- and second-emitting neutrons are detected, while the third-emitting neutron is undetected when the $(\gamma,3n)$ reaction takes place. The single neutron event consists of six terms in which the third term means that the second-emitting neutron is detected, while the first-emitting neutron is undetected when the $(\gamma,2n)$ reaction takes place. Note that kinetic energy of the j -th emitting neutron is, in principle, different from that of the k -th ($k \neq j$) emitting neutron. Thus, the RR technique, which is applied to the experimental quantities, N_s , N_d , and N_i , does not work to determine neutron kinetic energies.

One can immediately see that the only solution to determine the number of (γ, xn) reactions is to make the neutron detection efficiency (\mathcal{E}) independent of neutron kinetic energies. In this case, the neutron events are expressed as follows.

$$\begin{aligned} N_s &= N_1 \cdot \mathcal{E} + N_2 \cdot 2C_1 \cdot \mathcal{E} \cdot (1-\mathcal{E}) + \\ N_{3 \cdot 3} C_1 \cdot \mathcal{E} \cdot (1-\mathcal{E})^2 \end{aligned} \quad (4)$$

$$N_d = N_2 \cdot \mathcal{E}^2 + N_{3 \cdot 3} C_2 \cdot \mathcal{E}^2 \cdot (1-\mathcal{E}) \quad (5)$$

$$N_i = N_3 \cdot \mathcal{E}^3 \quad (6)$$

Now one can solve a set of Eqs. (4), (5), and (6) to determine N_1 , N_2 , and N_3 , the numbers of (γ, xn) reactions ($x=1,2,3$) induced by the γ -ray beam from the experimental quantities, N_s , N_d , and N_i . Note that \mathcal{E} is now a constant. It is straightforward to determine (γ, xn) cross sections (σ_{xn}) by $\sigma_{xn} = N_x / (N_\gamma \cdot N_T)$ with the number of γ -rays (N_γ) incident on target nuclei (N_T) per unit area.

We have developed a flat-response neutron detector (FRD) for (γ, xn) cross section measurements in the IAEA-CRP [9]. The detector consists of three concentric rings of 4, 9, and 18 ^3He counters embedded in a 46 cm (horizontally) x 46 cm (vertically) x 50 cm (along the beam axis) polyethylene moderator at the distances of 5.5, 13.0 and 16.0 cm from the γ -ray beam axis, respectively. Figure 3 shows a photograph of the flat-response detector.

The total and partial detection efficiencies of the FRD are shown in Fig. 4 along with results of the calibration with a ^{252}Cf source and GEANT4 simulation.

The emission rate of the ^{252}Cf source was calibrated at the National Metrology Institute of Japan. Results are plotted at the average energy ($3T/2$) of the ^{252}Cf neutron spectrum corresponding to the temperature $T = 1.42$ MeV [10], where the fission spectrum is given by the Maxwell-Boltzmann distribution, $P(E) \propto E^{1/2} \cdot \exp(-ET)$. The GEANT4 simulation was performed for monochromatic neutrons (dotted lines) and neutron-evaporation spectra (solid lines) expressed by $P(E) \propto E \cdot \exp(-ET)$ with the Weisskopf-Ewing model [11]. Results for the evaporation spectra are shown at the average neutron energy ($2T$). The total efficiency for the

evaporation spectra average over the energy range of 0.01 – 5.0 MeV is 36.5 % with one standard deviation of 1.6%. The neutron energy range was investigated for $^{209}\text{Bi}(\gamma, xn)$ reactions at 40 MeV by means of a Monte Carlo statistical model calculation [12]. The calculation shows that the mean neutron energy for the (γ, xn) channel with $x=2-5$ lies within the energy range.

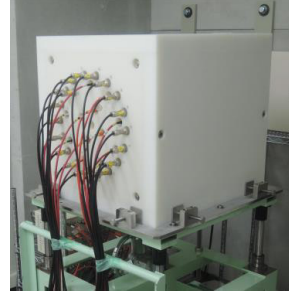


Fig. 3 (color online) Flat-response neutron detector.

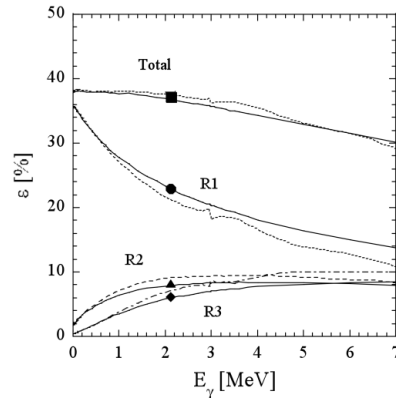


Fig. 4 Total and partial detection efficiencies of the flat-response neutron detector for neutron-evaporation spectra (solid lines) and monochromatic neutrons (dotted lines).

The neutron angular distribution effect on the detection efficiency was investigated for isotropic s- and anisotropic p-wave neutrons with respect to the linear polarization of the laser Compton-scattering γ -ray beam. It was confirmed that the FRD has nearly identical efficiencies for s- and p-wave neutrons, where the strong spatial anisotropy in the p-wave neutron emission is smeared out during the thermalization of neutrons in the moderator. This is a common feature of moderation-based neutron detectors of the Livermore type.

Figure 5 shows distributions of the arrival-time of neutrons at the FRD for single, double, and triple neutron events in the $^{209}\text{Bi}(\gamma, xn)$ reaction at 34 MeV. Time distributions of the 1st- and 2nd-arriving neutrons and the 1st-, 2nd-, and 3rd-arriving neutrons are also shown for the double and triple neutron events, respectively. It was found that the sum distribution is

best fitted with a linear combination of two exponential functions with time constants, $\sim 10\mu\text{s}$ for the fast component and $\sim 100\mu\text{s}$ for the slow component, respectively. One can see that the major part of the 1st-arriving neutrons is well characterized by the fast component, while the 2nd and 3rd components by the slow component. The fast component may represent neutron detection with the first ring located at 5.5cm, while the slow component neutron detection with the second and third rings at 13 and 16cm. The arrival-time distribution is a good subject to investigate by a Monte Carlo simulation.

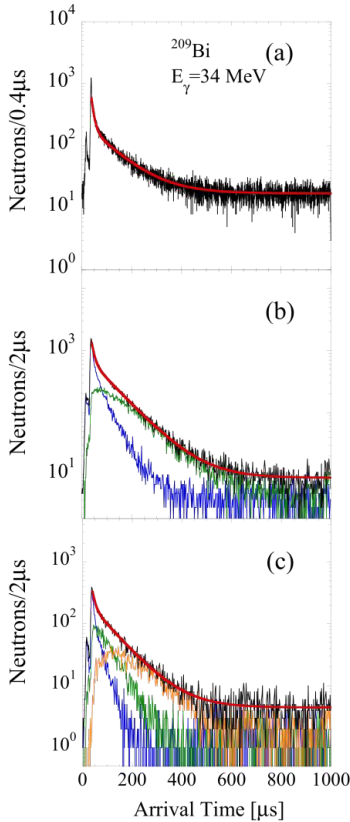


Fig. 5 (color online) Arrival-time distributions of neutrons detected with the flat-response detector for single (a), double (b), and triple (c) neutron events in the $^{209}\text{Bi}(\gamma,xn)$ reactions at 34 MeV. For the double neutron events, the arrival-time distribution is shown for the 1st- (blue) and 2nd-arriving (green) neutrons as well as the sum neutrons. Similarly, for the triple neutron events, the arrival-time distribution is shown for the 1st- (blue), 2nd- (green), and 3rd-arriving (orange) neutrons as well as the sum neutrons. The best fits to the single (a), double sum (b), and triple sum events (c) are shown by the red lines.

3 New $^{209}\text{Bi}(\gamma,xn)$ data

The technique of the direct neutron-multiplicity sorting was first used to measure $^{209}\text{Bi}(\gamma,xn)$ cross sections [13] in the IAEA-CRP. Figure 6 shows partial photoneutron cross sections obtained in the new measurement in comparison with the TALYS [14] calculation of the giant-dipole resonance (GDR) and quasi-deuteron component [15,16]. The GDR cross section was obtained within the Hartree-Fock-Bogolyubov plus quasiparticle random phase approximation approach on the basis of the DIM Gogny interaction both for the E1 [17] and M1 [18] components. See Ref. [13] for details.

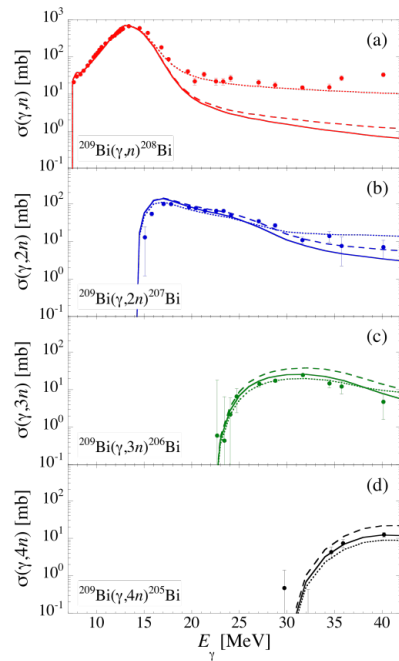


Fig. 6 (color online) Comparison with the present partial photoneutron cross sections with the TALYS calculation with the default preequilibrium parametrization (solid lines), the modified Levinger parameter $L=20$ with (dotted lines) and without (dashed lines) surface effects ($E_{\text{surf}}=1$ MeV).

3.1 Puzzle of the $^{209}\text{Bi}(\gamma,1n)$ cross section

While the (γ,xn) cross sections with $x=2, 3$, and 4 are reasonably well reproduced by the TALYS calculation with the default preequilibrium parametrization (solid lines), the large $(\gamma,1n)$ cross section above 25 MeV is highly puzzling. The standard statistical model predicts small $(\gamma,1n)$ cross sections (3–4 mb) in this energy range. Furthermore, it is kinematically considered that kinetic-energies of neutrons emitted in the $(\gamma,1n)$ channel are very large; they are outside the energy range (0.01 – 5.0

MeV) in which the flat-response is assured.

Although the (γ, In) cross section can be artificially enhanced by tuning the Levinger parameter involved in the quasi-deuteron model or even by tuning the statistical model parameters within a reasonable range, the large (γ, In) cross section is most likely experimental; it may be caused by secondary γ -rays produced by electrons and positrons resulting from the pair production and Compton scattering of the γ -ray beam in passing through a thick (10cm) ^{209}Bi target. A simulation shows that contaminant (γ, In) reactions are induced by secondary γ -rays most effectively around the peak of the GDR, thus significantly reducing the relevant (γ, In) cross section at high energies. We experimentally investigate this effect by changing the thickness of ^{209}Bi targets. We will write an erratum paper after a thorough investigation in experiment and simulation.

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