

ADAPTIVE SOLUTION OF THE NEUTRON DIFFUSION EQUATION WITH HETEROGENEOUS COEFFICIENTS USING THE MIXED FINITE ELEMENT METHOD ON STRUCTURED MESHES

Minh-Hieu DO^{1*}, Patrick CIARLET², and François MADIOT¹

¹ DEN-Service d'études des réacteurs et de mathématiques appliquées (SERMA)
CEA, Université Paris-Saclay, F-91191, Gif-sur-Yvette, France.

²Poems, UMA, ENSTA Paristech, 828 Boulevard des Maréchaux, 91120 Palaiseau.
minh-hieu.do@cea.fr, patrick.ciarlet@ensta-paris.fr, francois.madiot@cea.fr

ABSTRACT

The neutron transport equation can be used to model the physics of the nuclear reactor core. Its solution depends on several variables and requires a lot of high precision computations. One can simplify this model to obtain the SP_N equation for a generalized eigenvalue problem. In order to solve this eigenvalue problem, we usually use the inverse power iteration by solving a source problem at each iteration. Classically, this problem can be recast in a mixed variational form, and then discretized by using the Raviart-Thomas-Nédélec Finite Element. In this article, we focus on the steady-state diffusion equation with heterogeneous coefficients discretized on Cartesian meshes. In this situation, it is expected that the solution has low regularity. Therefore, it is necessary to refine at the singular regions to get better accuracy. The Adaptive Mesh Refinement (AMR) is one of the most effective ways to do that and to improve the computational time. The main ingredient for the refinement techniques is the use of a posteriori error estimates, which gives a rigorous upper bound of the error between the exact and numerical solution. This indicator allows to refine the mesh in the regions where the error is large. In this work, some mesh refinement strategies are proposed on the Cartesian mesh for the source problem. Moreover, we numerically investigate an algorithm which combines the AMR process with the inverse power iteration to handle the generalized eigenvalue problem.

KEYWORDS: Neutronics, diffusion equation, eigenvalue problem, a posteriori error estimates, adaptive mesh refinement.

1. Introduction

Numerical simulations of nuclear reactor core are generally expensive as they require the exact solution of the neutron transport equation. These simulations are computationally expensive since one has to deal with many variables such as the neutron position in space, the motion direction and neutron energy. In this work, we use the multi-group theory for the discretization of the energy

*Corresponding author

variable and we focus on the steady state multi-group neutron diffusion equation. However, we choose to present the results in the form of the multi-group SP_N equation since this extension is straightforward. In the time independent case, the multi-group SP_N equation corresponds to a generalized eigenvalue problem:

$$\begin{cases} \mathbb{T}_o \mathbf{p} + \mathbf{grad} (\mathbb{H}\phi) = 0, & \text{in } \mathcal{R}, \\ \mathbb{H}^\top \mathbf{div} (\mathbf{p}) + \mathbb{T}_e \phi = \frac{1}{k_{\text{eff}}} \mathbb{M}_f \phi, & \text{in } \mathcal{R}, \\ \phi|_{\partial\mathcal{R}} = 0, & \text{on } \partial\mathcal{R}, \end{cases} \quad (1)$$

where \mathcal{R} is a bounded and open subset of \mathbb{R}^d with $d = 1, 2, 3$. Let G be the number of energy group and N an odd number which represents for the order of the SP_N equation. The number of odd and even moments is $\hat{N} = \frac{N+1}{2}$. The vector ϕ and \mathbf{p} represent the even and odd components of the multi-group flux defined by:

$$\begin{aligned} \phi &= (\phi^1, \dots, \phi^G)^\top \in \mathbb{R}^{\hat{N} \times G} & \text{with } \phi^g &= (\phi_0^g, \phi_2^g, \dots, \phi_{N-1}^g)^\top \in \mathbb{R}^{\hat{N}}, \\ \mathbf{p} &= (\mathbf{p}^1, \dots, \mathbf{p}^G)^\top \in \mathbb{R}^{d \times \hat{N} \times G} & \text{with } \mathbf{p}^g &= (\mathbf{p}_1^g, \mathbf{p}_3^g, \dots, \mathbf{p}_N^g)^\top \in \mathbb{R}^{d \times \hat{N}}. \end{aligned}$$

The operators \mathbb{T}_o , \mathbb{T}_e , \mathbb{M}_f and \mathbb{H} are defined by block matrices:

$$\begin{aligned} (\mathbb{T}_o)_{g,g'} &= \begin{cases} \mathbb{T}_o^g & \text{for } g = g' \\ -\mathbb{S}_o^{g' \rightarrow g} & \text{for } g \neq g' \end{cases}, & (\mathbb{T}_e)_{g,g'} &= \begin{cases} \mathbb{T}_e^g & \text{for } g = g' \\ -\mathbb{S}_e^{g' \rightarrow g} & \text{for } g \neq g' \end{cases}, \\ (\mathbb{M}_f)_{g,g'} &= \chi^g \mathbb{M}_f^{g'}, & (\mathbb{H})_{g,g'} &= \delta_{g,g'} \mathbb{H}^g. \end{aligned}$$

In the above definition, χ^g is the fission spectrum and the blocks \mathbb{T}_o^g , \mathbb{T}_e^g , $\mathbb{S}_o^{g' \rightarrow g}$, $\mathbb{S}_e^{g' \rightarrow g}$, $\mathbb{M}_f^{g'}$, \mathbb{H}^g are given by

$$\begin{aligned} \mathbb{T}_o^g &= \text{diag} \left[(t_{2n+1} \Sigma_{r,2n+1}^g)_{n=0}^{\hat{N}} \right], & \mathbb{T}_e^g &= \text{diag} \left[(t_{2n} \Sigma_{r,2n}^g)_{n=0}^{\hat{N}} \right], & \mathbb{S}_o^{g' \rightarrow g} &= \text{diag} \left[(t_{2n+1} \Sigma_{s,2n+1}^{g' \rightarrow g})_{n=0}^{\hat{N}} \right], \\ \mathbb{S}_e^g &= \text{diag} \left[(t_{2n} \Sigma_{s,2n}^{g' \rightarrow g})_{n=0}^{\hat{N}} \right], & \mathbb{M}_f^{g'} &= \left(t_0 \nu^{g'} \Sigma_f^{g'} \delta_{2m,0} \delta_{2n,0} \right)_{n,m=0}^{\hat{N}}, & \mathbb{H}^g &= (\delta_{n,m} + \delta_{n,m-1})_{n,m=0}^{\hat{N}} \end{aligned}$$

where Σ_t^g is the total cross section, $\Sigma_s^{g' \rightarrow g}$ is the scattering cross section and the removal macroscopic cross sections defined by $\Sigma_{r,n}^g(\mathbf{x}) = \Sigma_{t,n}^g(\mathbf{x}) - \Sigma_{s,n}^{g \rightarrow g}(\mathbf{x})$. The notation t_n stands for the normalization factor defined by $t_0 = 1$ and $t_n = \frac{4n^2 - 1}{nt_{n-1}}$ for $n > 0$. The notation ν^g is the number of neutron emitted per fission, and Σ_f^g is the macroscopic fission cross section. The multiplication factor k_{eff} is the inverse of the eigenvalue and characterizes the physical state of the core reactor. In this work, we focus on the development of the project APOLLO3, a shared platform among CEA, FRAMATOME and EDF, which includes different deterministic solvers for the neutron transport equation. Particularly, we are interested in the MINOS solver based on the mixed Raviart-Thomas-Nedelec finite element method and implemented on Cartesian and hexagonal grids for the multi-group SP_N equation [1]. Since the removal matrices \mathbb{T}_o and \mathbb{T}_e are usually heterogeneous (piecewise constant), the solution of the SP_N equation may have some singularities which limit the precision and convergence of the solution [2]. It is well known that the mesh subdivision method is one of the most effective ways to treat this problem [3–5].

The purpose of this work is to develop an adaptive solution for the eigenvalue problem. In particular, this article is organized as follows. In Section 2, we present the Adaptive Mesh Refinement (AMR) strategies for the source problem by using a posteriori error estimators. Some marker cell strategies on Cartesian mesh are also detailed in this section. Next, we propose a splitting algorithm for the eigenvalue problem in Section 3. Some numerical test cases are shown in Section 4 to illustrate our purposes. Finally, concluding remarks in Section 5 complete the study.

2. Adaptive mesh refinement strategies

To be convenient, let us introduce the following function spaces: $\mathbf{Q} = \mathbf{H}(\text{div}, \mathcal{R})^{\hat{N} \times G}$, $V = H_0^1(\mathcal{R})^{\hat{N} \times G}$, $V' = H^{-1}(\mathcal{R})^{\hat{N} \times G}$, $L = L^2(\mathcal{R})^{\hat{N} \times G}$, $\mathbf{X} = \mathbf{Q} \times L$ and the following operators:

$$\circ : \begin{cases} \mathbb{R}^{\hat{N} \times G} \times \mathbb{R}^{\hat{N} \times G} \rightarrow \mathbb{R} \\ (x, y) \rightarrow \sum_{g=1}^G \sum_{n=1}^{\hat{N}} x_n^g y_n^g \end{cases}, \quad \odot : \begin{cases} (\mathbb{R}^d)^{\hat{N} \times G} \times (\mathbb{R}^d)^{\hat{N} \times G} \rightarrow \mathbb{R} \\ (\mathbf{x}, \mathbf{y}) \rightarrow \sum_{g=1}^G \sum_{n=1}^{\hat{N}} \mathbf{x}_n^g \cdot \mathbf{y}_n^g. \end{cases}$$

Let \mathbf{X}_h be the discrete space associated to the mixed Raviart-Thomas finite elements. We denote $\zeta_h = (\mathbf{p}_h, \phi_h) \in \mathbf{X}_h$ and $\xi_h = (\mathbf{q}_h, \psi_h) \in \mathbf{X}_h$. The discrete variational formulation associated to Problem (1) reads:

$$\text{Find } (\zeta_h, k_{\text{eff}}^h) \in \mathbf{X}_h \times \mathbb{R}^+ \text{ such that for all } \xi_h \in \mathbf{X}_h, B(\zeta_h, \xi_h) = \frac{1}{k_{\text{eff}}^h} f(\zeta_h, \xi_h), \quad (2)$$

where the bilinear form $B(\zeta_h, \xi_h)$ and $f(\zeta_h, \xi_h)$ are respectively defined by

$$B(\zeta_h, \xi_h) := \int_{\mathcal{R}} \mathbb{T}_o \mathbf{p}_h \odot \mathbf{q}_h - \int_{\mathcal{R}} \phi_h \circ \mathbb{H}^\top \text{div } \mathbf{q}_h + \int_{\mathcal{R}} \psi_h \circ \mathbb{H}^\top \text{div } \mathbf{p}_h + \int_{\mathcal{R}} \mathbb{T}_e \phi_h \circ \psi_h,$$

$$f(\zeta_h, \xi_h) := \int_{\mathcal{R}} \mathbb{M}_f \phi_h \circ \psi_h.$$

The inverse power iteration algorithm is a well known method for the resolution of eigenvalue problem. At iteration $m + 1$, we deduce $(\mathbf{p}^{m+1}, \phi^{m+1}, k_{\text{eff}}^{m+1})$ from $(\mathbf{p}^m, \phi^m, k_{\text{eff}}^m)$ by solving the following source problem:

$$\begin{cases} \mathbb{T}_o \mathbf{p}^{m+1} + \mathbf{grad}(\mathbb{H} \phi^{m+1}) = 0, & \text{in } \mathcal{R}, \\ \mathbb{H}^\top \text{div}(\mathbf{p}^{m+1}) + \mathbb{T}_e \phi^{m+1} = S_m, & \text{in } \mathcal{R}, \\ \phi^{m+1}|_{\partial \mathcal{R}} = 0, & \text{on } \partial \mathcal{R}, \end{cases} \quad (3)$$

where the source term is defined by $S_m = (k_{\text{eff}}^m)^{-1} \mathbb{M}_f \phi^m$. This iterative process is stopped when the relative error between the current and previous iteration of the flux as well as the multiplication factor k_{eff} are smaller than the given threshold numbers. In this section, we aim to illustrate the adaptive mesh refinement strategies for the source problem (3). In general, the h-refinement is to generate a sequence of mesh \mathcal{T}_{h_k} from the initial mesh \mathcal{T}_{h_0} by using the AMR strategy which is in general an iterative loop where at each iteration, we consider the four modules:

SOLVE \rightarrow **ESTIMATE** \rightarrow **MARK** \rightarrow **REFINE**.

In particular, the module **SOLVE** is the mixed Raviart-Thomas finite element discretization on the mesh \mathcal{T}_{h_k} of the source problem (3). In module **ESTIMATE**, the η_K local error indicator on each element is obtained from a posteriori error estimate for the discrete solution. The purpose of the module **MARK** is to select a set of elements with large error to be refined. In the module **REFINE**, we use some rule for dividing the marked cells such that the obtained mesh $\mathcal{T}_{h_{k+1}}$ is still conforming. We now detail how we obtain the error indicators for the source problem and some marker cell strategies on Cartesian mesh for the MINOS solver.

2.1. A posteriori error estimate for the source problem

Let us define the linear form $f_S(\xi) := \int_{\mathcal{R}} S_f \circ \psi_h$ with a given source term $S_f \in V'$. The variational formulation of the source problem writes,

$$\text{Find } \zeta = (\mathbf{p}, \phi) \in \mathbf{X} \text{ such that for all } \xi = (\mathbf{q}, \psi) \in \mathbf{X}, B(\zeta, \xi) = f_S(\xi) \quad (4)$$

The associated discrete variational formulation reads,

$$\text{Find } \zeta_h = (\mathbf{p}_h, \phi_h) \in \mathbf{X}_h \text{ such that for all } \xi_h = (\mathbf{q}_h, \psi_h) \in \mathbf{X}_h, B(\zeta_h, \xi_h) = f_S(\xi_h). \quad (5)$$

We now define the norm on \mathbf{X} by $\|\zeta\|_S^2 := \sum_{K \in \mathcal{T}_h} \|d\mathbb{T}_o^{1/2} \mathbf{p}\|_K^2 + \|d\mathbb{T}_e^{1/2} \phi\|_K^2 + \|d\mathbb{T}_e^{-1/2} \mathbb{H}^\top \text{div } \mathbf{p}\|_K^2$

where the notation $d\mathbb{T}$ stands for the diagonal of the matrix \mathbb{T} . Finally, let us define the following norm

$$|\zeta|_+ = \sup_{\xi \in \mathbf{X}, \|\xi\|_S \leq 1} B(\zeta, \xi).$$

In the mixed dual Raviart-Thomas finite element method, we only have $\phi_h \in L_h$. In particular, the discrete flux is a priori not in V . In order to find a posteriori error estimate for the source problem, it is crucial to introduce a discrete potential reconstruction flux $\tilde{\phi}_h \in V$ by using the average operator (see [6] and references therein). Next, let ζ and ζ_h be respectively the solution of (4) and (5). Let $\tilde{\zeta}_h = (\mathbf{p}_h, \tilde{\phi}_h)$ be a reconstruction of ζ_h in $\mathbf{Q} \times V$. For any $K \in \mathcal{T}_h$, we define respectively the residual estimator, flux estimator, symmetric non-conforming estimator and non-symmetric non-conforming estimator by:

$$\eta_{R,K} = \left\| d\mathbb{T}_e^{-1/2} (S_f - \mathbb{H}^\top \text{div}(\mathbf{p}_h) - \mathbb{T}_e \phi_h) \right\|_K, \quad \eta_{F,K} = \left\| d\mathbb{T}_o^{-1/2} \left(\mathbb{T}_o \mathbf{p}_h + \mathbf{grad}(\mathbb{H} \tilde{\phi}_h) \right) \right\|_K$$

$$\eta_{NC,K} = \left\| d\mathbb{T}_e^{1/2} (\tilde{\phi}_h - \phi_h) \right\|_K, \quad \eta_{NC^*,K} = \left\| d\mathbb{T}_e^{-1/2} \mathbb{T}_e (\tilde{\phi}_h - \phi_h) \right\|_K.$$

Then it stands

$$|\zeta - \zeta_h|_+ \leq \left(\sum_{K \in \mathcal{T}_h} \eta_{R,K}^2 + \eta_{F,K}^2 \right)^{1/2} + \left(\sum_{K \in \mathcal{T}_h} \eta_{NC,K}^2 \right)^{1/2} + 2 \left(\sum_{K \in \mathcal{T}_h} \eta_{NC^*,K}^2 \right)^{1/2}. \quad (6)$$

The proof of this results for the multi-group SP_N equation is an extension of the one for the diffusion one group equation proposed in [6, Chapter 8]. For the sake of brevity, details of the numerical analysis for the diffusion equation are given in the companion paper [7].

2.2. Refinement strategies on Cartesian mesh

The AMR method is usually applied to the non-structured mesh like triangular grids and the marker strategy is based on the total error indicator on each cell $\eta_K = \sqrt{\eta_{R,K}^2 + \eta_{F,K}^2 + \eta_{NC,K}^2 + 4\eta_{NC^*,K}^2}$. For example, the classical Dorfler's marking strategy aims to select the minimal cardinality of the subset S_{h_k} such that $\sum_{K \in S_{h_k}} \eta_K \geq \theta \sum_{K \in \mathcal{T}_{h_k}} \eta_K$ for a fixed threshold number $0 < \theta \leq 1$. In this article, we focus on the solvers which use the structured meshes like the MINOS solver. In this case, to ensure the conformity of the mesh, it is essential to refine the mesh according to the whole line in each direction (x, y, z) which contain the selected cells. As a consequence, it is obvious to

see that we use the error indicator of just some selected cells to refine the other cells located in the same line for a given direction. Therefore, it is extremely important to point out some other marker cell strategies based on more information. Instead of using the classical bulk chasing (Dorfler's marking strategy) defined on cell, we modify it to propose the construction of some other error indicators according to the lines of each direction and also on the "cross" value (the total error indicators of all the lines). In order to preserve Cartesian mesh structure, we just simply refine the selected lines by dividing into two lines.

3. Splitting algorithm for the generalized eigenvalue problem

In order to combine the AMR process with the power inverse iteration, a first approach is the combining algorithm where at each outer iteration, the AMR process is taken into account for the source problem [6, Chapter 8]. In this work, we would like to investigate the splitting algorithm is illustrated in Algorithm 1: The first process is the power inverse iteration for the eigenvalue problem and the second one is the AMR strategy for the source problem. The implementation of the splitting algorithm is easier than the combining one since we can use the solver of the problem (2) as a black box. The stopping criterion of the algorithm is usually defined by the maximum of the error indicators. However, for comparison purposes, if we have the reference value of the multiplication factor k_{eff} , we can define another stopping criterion based on the relative error given by $\varepsilon_{k_{\text{eff}}} = \frac{|k_{\text{eff}}^h - k_{\text{eff}}|}{k_{\text{eff}}}$ where k_{eff}^h is the numerical multiplication factor.

Algorithm 1: The splitting algorithm

Input: The threshold numbers \mathcal{E}_{AMR} or $\mathcal{E}_{\text{Accuracy}}$ and the maximum of the refinement n_{max}

Output: The solution $(\mathbf{p}_{h_k}, \phi_{h_k}, k_{\text{eff}}^{h_k})$ associated to the mesh \mathcal{T}_{h_k} .

```

1 begin
2   Set  $k \leftarrow 0$  and the initial mesh  $\mathcal{T}_{h_k}$ 
3   Solve Problem (2) on mesh  $\mathcal{T}_{h_k}$  by the power inverse iteration
4   Set the counter refinement parameter  $n = 0$ 
5   Solve Problem (5) on the mesh  $\mathcal{T}_{h_{k+n}}$  by using module SOLVE
6   Evaluate the local error indicator  $\eta_K$  in the module ESTIMATE
7   if  $\max(\eta_K) > \mathcal{E}_{\text{AMR}}$  or  $\varepsilon_{k_{\text{eff}}} > \mathcal{E}_{\text{Accuracy}}$  then
8     | Select the minimal subset  $S_{h_{k+n}}$  of  $\mathcal{T}_{h_{k+n}}$  with the help of the module MARK
9     | Using module REFINE to obtain a new mesh  $\mathcal{T}_{h_{k+n+1}}$  and then  $n \leftarrow n + 1$ 
10  else
11    | Stop the algorithm
12  end
13  if  $n \leq n_{\text{max}}$  then
14    | Go to line 5
15  else
16    | Set  $k \leftarrow k + n$  and go to line 3
17  end
18  Return the solution  $(\mathbf{p}_{h_k}, \phi_{h_k}, k_{\text{eff}}^{h_k})$ 
19 end

```

4. Numerical results

4.1. The one group case

We present here a checkerboard test case for the one group diffusion equation on the domain of the computation $\mathcal{R} = (0, 100)^2$. The diffusion coefficient $D = (3\Sigma_t)^{-1}$ is given as in Figure 1(a) with $D = 5$ in ■ region and $D = 1$ in ■ region. The fission cross section $\Sigma_f = 1$ and the absorption cross section $\Sigma_a = \Sigma_t - \Sigma_s = 1$. This test aims at investigating the effect of some marker strategies in the discussion above. Therefore, we compare these marker strategies for the splitting algorithm with $n_{\max} = 1$, $\theta = 0.5$ and the lowest order Raviart-Thomas finite element RT_0 . We fix $\mathcal{E}_{\text{Accuracy}} = 10^{-5}$ and the reference eigenvalue is $k_{\text{reff}} = 0.995194$ which is obtained from the computation on the uniform mesh 1000×1000 . Figure 1(b) clearly shows that the direction marker is much better than the element and cross marker method since it requires less elements to reach the target accuracy. It was observed that the uniform mesh needs 32400 elements to obtain the same accuracy. Moreover, Figure 1(c)-1(e) illustrate that there are more refinement at singularities with the direction marker strategy which helps to improve the accuracy of the solution.

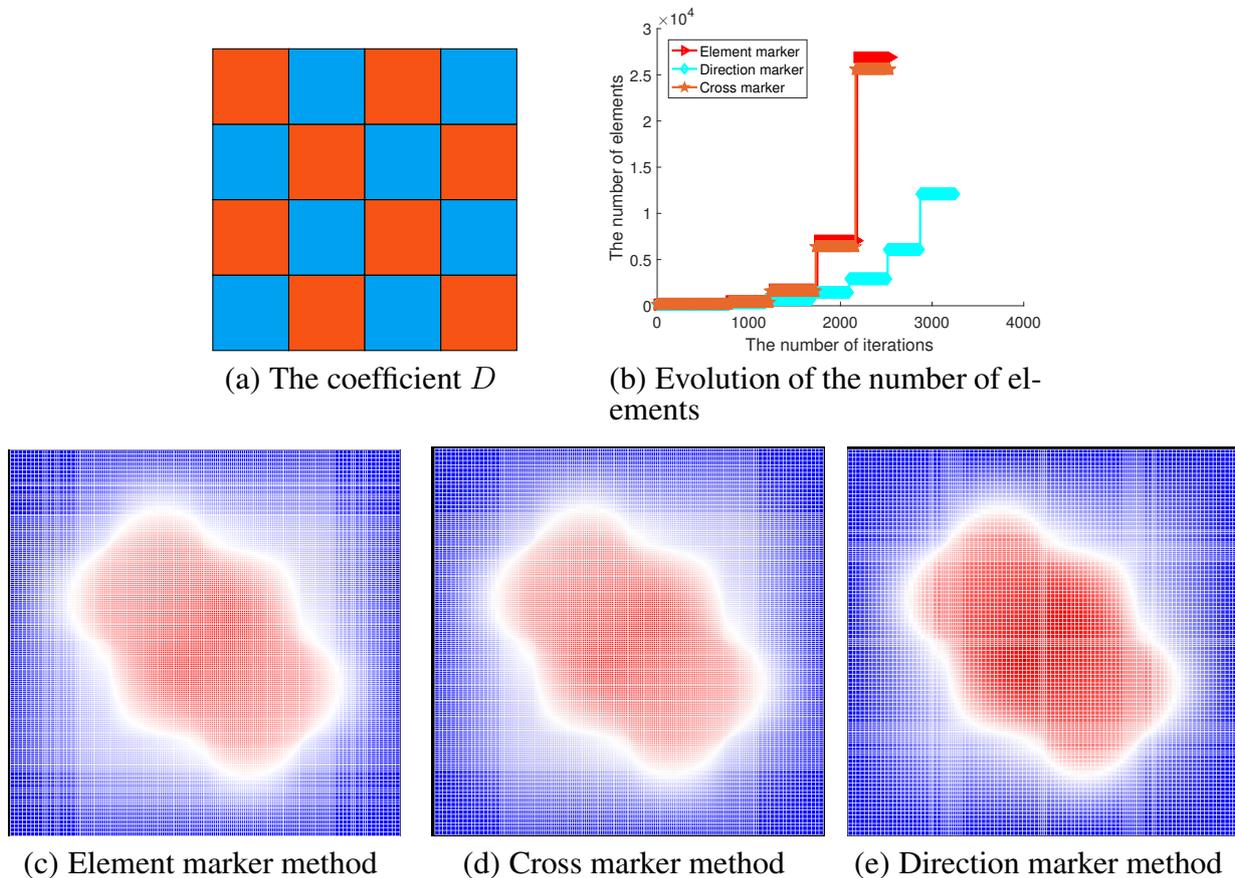


Figure 1: The diffusion coefficient, the evolution of the total number of element during the splitting algorithm and the discrete solution ϕ_h on refined mesh.

4.2. The multi-group case

We consider here the small FBR core proposed in [8] with 4 energy groups and where the control rod is withdrawn (model 2-case 1). However, we impose zero flux boundary condition on the boundary $\partial\mathcal{R}$. Figure 2 shows the behavior of all error components. As can be seen, the structures of $\eta_{NC^*,K}$ and the total error η_K are very similar. Therefore, the non-conforming estimator $\eta_{NC^*,K}$ is the main contributor to the error for this benchmark test case. Figure 3 shows some refined meshes of the refinement process.

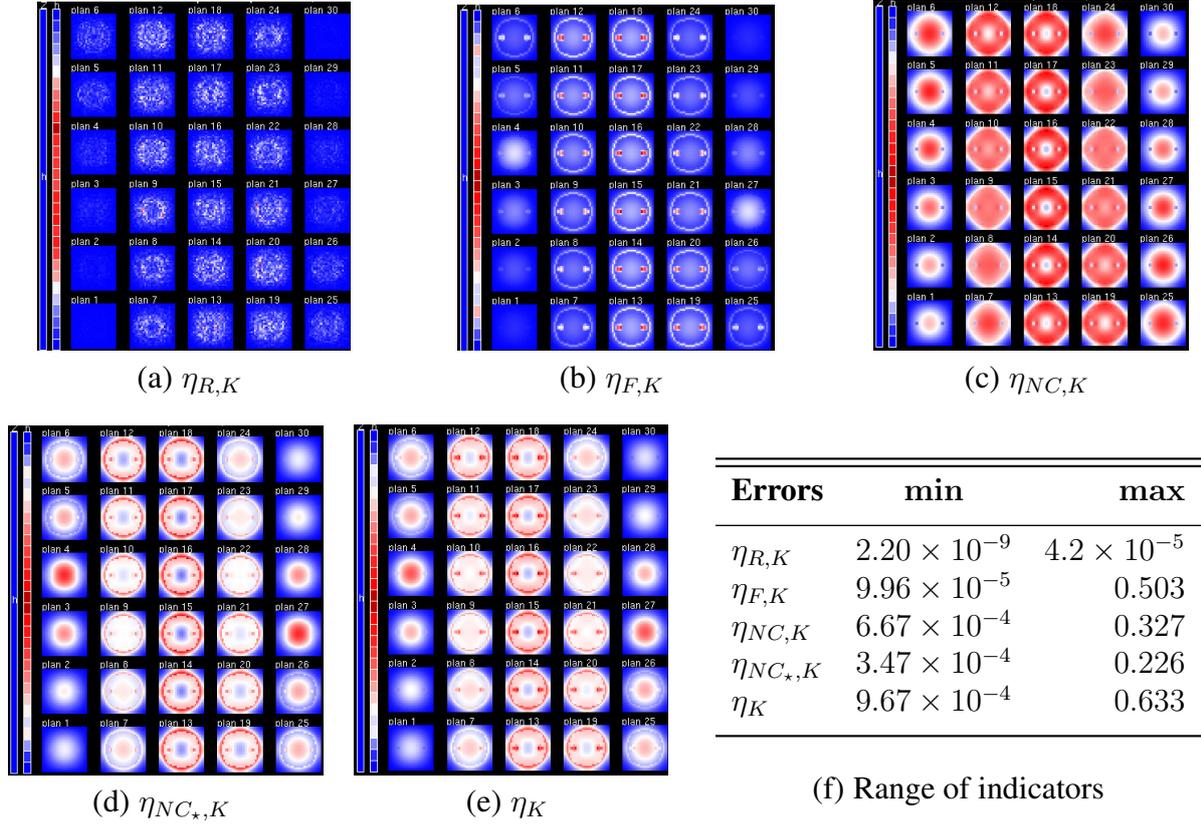


Figure 2: Error indicators with RT_0 .

Table 1: Data of adaptive mesh refinement by using direction marker method

Iteration	Number of elements	$\max_{K \in \mathcal{T}_h} \eta_K$	k_{eff}	$ \mathcal{R} / \min_{K \in \mathcal{T}_h} K $
0	23520	0.632849	0.96405	23520
1	67200	0.321258 ($\max \eta_K^0 / 2$)	0.964081	188160
2	205204	0.119536 ($\max \eta_K^1 / 2.7$)	0.963956	1.50528×10^6
3	599343	0.053853 ($\max \eta_K^2 / 2.2$)	0.963818	1.20422×10^7
4	1813560	0.028126 ($\max \eta_K^3 / 1.9$)	0.963686	1.20422×10^7
5	5110200	0.011569 ($\max \eta_K^4 / 2.4$)	0.963669	9.6338×10^7

Since there is no analytical value for k_{eff} , the stopping criterion is given by $\mathcal{E}_{AMR} = 0.02$. The numerical multiplication factor and the total number of element are given in Table 1.

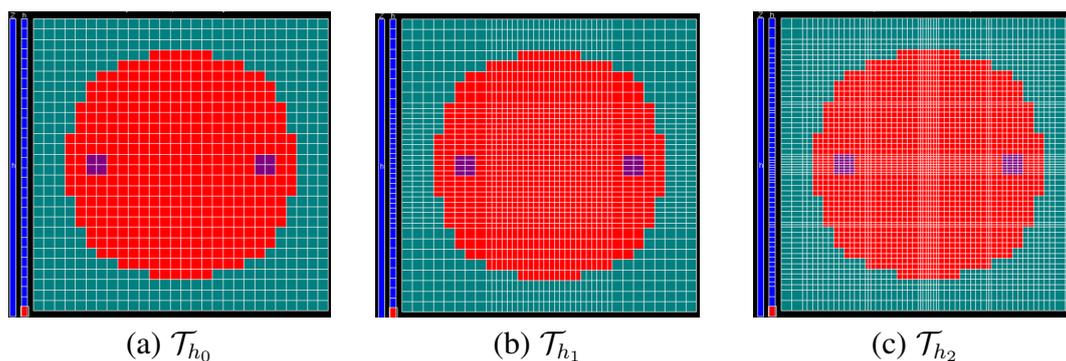


Figure 3: Adaptive meshes with direction marker method.

5. Conclusion

In this work, we propose an adaptive mesh refinement strategy for the neutron diffusion equation which relies on a posteriori error estimators for the source problem and a splitting approach to handle the generalized eigenvalue problem. Some marker cell strategies on the Cartesian mesh are also investigated and the direction marker method is the best candidate. Future work will be dedicated to the coupling between a posteriori error estimate and non-matching grid domain decomposition method to have locally refinement one each subdomain.

REFERENCES

- [1] A.-M. Baudron and J.-J. Lautard. “MINOS: a simplified PN solver for core calculation.” *Nuclear Science and Engineering*, **volume 155**(2), pp. 250–263 (2007).
- [2] P. Ciarlet Jr, L. Giret, E. Jamelot, and F. Kpadonou. “Numerical analysis of the mixed finite element method for the neutron diffusion eigenproblem with heterogeneous coefficients.” *ESAIM: Mathematical Modelling and Numerical Analysis*, **volume 52**(5), pp. 2003–2035 (2018).
- [3] Y. Wang, W. Bangerth, and J. Ragusa. “Three-dimensional h-adaptivity for the multigroup neutron diffusion equations.” *Progress in Nuclear Energy*, **volume 51**(3), pp. 543–555 (2009).
- [4] Y. Wang and J. Ragusa. “Application of hp adaptivity to the multigroup diffusion equations.” *Nuclear Science and Engineering*, **volume 161**(1), pp. 22–48 (2009).
- [5] M. Vohralík. “A posteriori error estimates for lowest-order mixed finite element discretizations of convection-diffusion-reaction equations.” *SIAM Journal on Numerical Analysis*, **volume 45**(4), pp. 1570–1599 (2007).
- [6] L. Giret. *Numerical Analysis of a Non-Conforming Domain Decomposition for the Multigroup SPN Equations*. Ph.D. thesis, University Paris-Saclay (2018).
- [7] P. Ciarlet Jr, M.-H. Do, and F. Madiot. “A posteriori error estimates of the Neutron Diffusion equations for mixed finite element discretizations.” *In preparation*.
- [8] T. Takeda and H. Ikeda. “3-D neutron transport benchmarks.” *Journal of Nuclear Science and Technology*, **volume 28**(7), pp. 656–669 (1991).